Florida Power
A Progress Energy Company

Crystal River Nuclear Plant
Docket No. 50-302
Operating License No. DPR-72

Ref: 10 CFR 50.54(f)

November 26, 2003
3F1103-05

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
11555 Rockville Pike
Rockville, MD 20852

Subject: Crystal River Unit 3 – Response to Item 2 of Bulletin 2002-01, "Reactor Pressure Vessel Head Degradation and Reactor Coolant Pressure Boundary Integrity" (TAC No. MB4539)

References: 1. FPC to NRC letter, 3F0302-11, dated March 28, 2002, Response to NRC Bulletin 2002-01, "Reactor Pressure Vessel Head Degradation and Reactor Coolant Pressure Boundary Integrity"

2. PEF to NRC letter, 3F0203-07, dated February 26, 2003, Crystal River Unit 3 - Twenty-Day Response to Order for Establishing Interim Inspection Requirements for Reactor Pressure Vessel Heads at Pressurized Water Reactors

Dear Sir:

Progress Energy Florida, Inc. (PEF) hereby submits the requested response to Item 2 of the subject Bulletin.

Request

Within 30 days after plant restart following the next inspection of the reactor pressure vessel head to identify any degradation, all PWR addressees are required to submit to the NRC the following information:

A. the inspection scope (if different than that provided in response to Item 1.D) and results, including the location, size, and nature of any degradation detected,

Response

Crystal River Unit 3 (CR3) achieved breaker closure and started up from Refueling Outage 13 (13R) on November 5, 2003. In the response to Item 1.D of Reference 1, CR3 committed to replace the Reactor Pressure Vessel head (RPVH). The commitment was fulfilled during 13R. The new RPVH closure head received full volumetric and visual baseline examinations. Subsequent examinations will be performed as stated in Reference 2.

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Progress Energy Florida, Inc.
Crystal River Nuclear Plant
15760 W. Powerline Street
Crystal River, FL 34428

Due to industry events concerning RPVH nozzle leakage, CR3 did perform a best effort visual exam (VT-2) of the accessible surfaces and nozzles of the RPVH prior to head replacement. This examination did not reveal any evidence of RPVH nozzle leakage.

Request

B. the corrective actions taken and the root cause of the degradation.

Response

During the Fall 2001 Refueling Outage 12 (12R), CR3 detected degradation of one Control Rod Drive Mechanism (CRDM) nozzle which was repaired during that outage. In addition to the corrective action to repair the CRDM nozzle in 12R, CR3 replaced the RPVH in 13R. The replacement RPVH has been designed to minimize the concerns for CRDM nozzle cracking and leakage associated with Primary Water Stress Corrosion Cracking (PWSCC) of the Alloy 600 nozzle material (root cause identified for the degradation of the CRDM nozzle repaired in 12R). Alloy 690 base and weld material has been used for the CR3 CRDM nozzles on the replacement RPVH.

No new regulatory commitments are made in this letter.

If you have any questions regarding this submittal, please contact Mr. Sid Powell, Supervisor, Licensing and Regulatory Programs at (352) 563-4883.

Sincerely,

Dale E. Young
Vice President
Crystal River Nuclear Plant

DEY/lvc

xc: NRR Project Manager
Regional Administrator, Region II
Senior Resident Inspector

STATE OF FLORIDA

COUNTY OF CITRUS

Dale E. Young states that he is the Vice President, Crystal River Nuclear Plant for Progress Energy Florida, Inc. (PEF); that he is authorized on the part of said company to sign and file with the Nuclear Regulatory Commission the information attached hereto; and that all such statements made and matters set forth therein are true and correct to the best of his knowledge, information, and belief.

Dale E. Young
Vice President
Crystal River Nuclear Plant

The foregoing document was acknowledged before me this 26th day of November, 2003, by Dale E. Young.

Signature of Notary Public
State of Florida

JANET SCHROEDER
MY COMMISSION # DD 128063
EXPIRES: June 20, 2006
Bonded Thru Notary Public Underwriters

(Print, type, or stamp Commissioned
Name of Notary Public)

Personally Known ✓ -OR- Produced Identification _____